

# Chandrasekhar-Kendall-Furth Configurations for Magnetic Confinement

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## 1. Chandrasekhar-Kendall-Furth (CKF) Force-Free Fields

A simply connected magnetic confinement scheme can be obtained superposing two axisymmetric homogeneous force-free fields, each with  $\vec{\nabla} \wedge \vec{B} = \mu \vec{B}$ , both having the same value of the relaxation parameter  $\mu = \mu_0 \vec{j} \cdot \vec{B} / B^2$ . The first is the Chandrasekhar-Kendall force-free field [1] of order-1, which in spherical geometry  $(r, \vartheta, \phi_G)$  admits the poloidal flux function:  $\psi_{\mu,1}^{CK}(r, \vartheta) = -(\mu r) j_1(\mu r) \sin \vartheta P_1^1(\cos \vartheta)$ , where  $j_1(\mu r)$  is the spherical Bessel function of order 1, having its  $m$ -th radial zero at  $(\mu r) = x_{1,m}$  and  $P_1^1(\cos \vartheta)$  is the Legendre polynomial of order 1. The second is the Furth square-toroid force-free field [2], which can be written as:  $\psi_{\mu,\lambda}^F(r, \vartheta) = \sqrt{\mu^2 - \lambda^2} r \sin \vartheta J_1(\sqrt{\mu^2 - \lambda^2} r \sin \vartheta) \cos(\lambda r \cos \vartheta)$ , for any value of  $\lambda < \mu$ , where  $J_1$  is the cylindrical Bessel function. The superposition of the two force-free fields is written:  $\psi(r, \vartheta) = \psi_{\mu,1}^{CK} + \gamma \psi_{\mu,\lambda}^F$  (Fig. 1). For values of the superposition constant  $\gamma \geq 0.402\dots$ , it contains, in a simply connected region near the origin, a toroidal current density  $j_\phi$  of the same sign and can be called a Chandrasekhar-Kendall-Furth force-free field (CKF).

Figure 2 shows the cross-section of the CKF force-free field with superposition constant  $\gamma = 0.55$  and details its composition in terms of different plasma regions, divided by a magnetic separatrix. The "main spherical torus" (ST) has a safety factor (inverse of the rotational transform,  $q = 1/t$ ), which is  $q_0^{ST} \sim 1.0$  on the magnetic axis and  $q_{95}^{ST} \sim 1.5$  at the edge (95% of the poloidal flux of the magnetic separatrix). The two "secondary tori" (SC), present on top and bottom of the main torus, also have  $q_0^{SC} \sim 1.0$  on their magnetic axes and  $q_{95}^{SC} \sim 1.5$  at their edges. The discharge surrounding the three tori, which will be dubbed as "spheromak" (P) has a larger safety factor, respectively  $q_0^P \sim 1.5$  on the symmetry axis and  $q_{95}^P \sim 3.7$  at the separatrix. When the superposition constant exceeds  $\gamma = 0.69\dots$ , the secondary tori disappear (see Fig. 1).

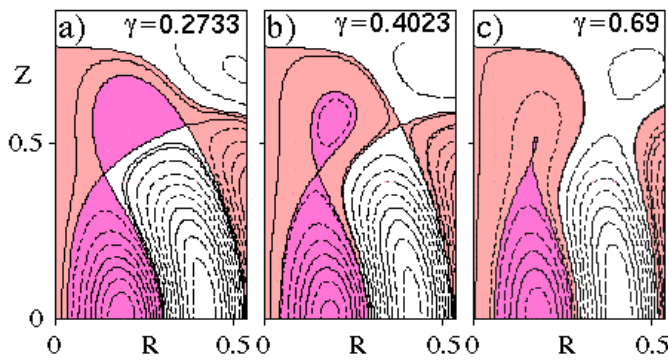


Fig. 1. Poloidal flux function contours of CKF force-free fields  $\psi = \psi_{\mu,1}^{CK} + \gamma \psi_{\mu,\lambda}^F$ , The superposition constants are: (a)  $\gamma = 0.273$ ; (b)  $\gamma = 0.402$ ; (c)  $\gamma = 0.690$ . Colored areas mean toroidal current density  $j_\phi > 0$ ; white areas mean  $j_\phi < 0$ .

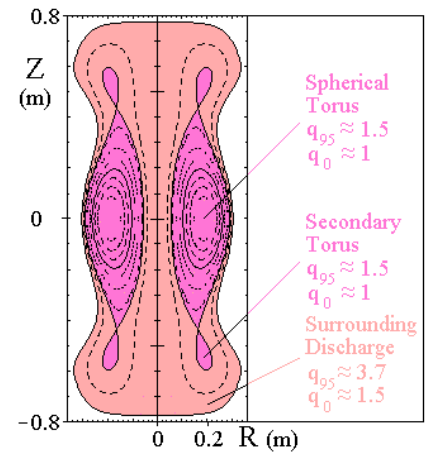


Fig. 2. Contours of the poloidal flux function of the Chandrasekhar-Kendall-Furth force-free field  $\psi = \psi_{\mu,1}^{CK} + \gamma \psi_{\mu,\lambda}^F$ , with superposition constant  $\gamma = 0.55$ .

## 2. Ideal MHD Stability of CKF Force-Free Fields

The ideal MHD stability of the Chandrasekhar-Kendall-Furth (CKF) force-free fields has been studied by solving the eigenvalue problem:  $\vec{W} \cdot \vec{\xi} = \omega^2 \vec{K} \cdot \vec{\xi}$ , where  $\vec{W}$  is the plasma perturbed potential energy and  $\vec{K}$  the plasma perturbed kinetic energy, associated with the perturbed plasma displacement  $\vec{\xi}$ , and  $\omega^2$  is the eigenvalue. The expressions for the perturbed energies become simpler [3] if the equilibrium is analyzed in non-orthogonal periodical Boozer coordinates ( $\psi_T$ -radial,  $\theta$ -poloidal,  $\phi$ -toroidal) [4], with Jacobian  $\sqrt{g} \propto 1/B^2$ . The radial variable  $\psi_T$  is the toroidal flux divided by  $2\pi$ , with  $\psi_T=0$  on the magnetic axis of the main torus,  $\psi_T=\psi_T^x$  on the separatrix and  $\psi_T=\psi_T^{\max}$  at the edge of the surrounding spheromak and on the symmetry axis. The continuity of the rotational transform  $t(\psi_T)$  and of the toroidal and poloidal plasma currents  $I(\psi_T)$  and  $f(\psi_T)$  joins the Boozer coordinates of these equilibria at the ST-P interface,  $\psi_T=\psi_T^x$  (see Fig. 3).

The energy principle for a compressible plasma [5] is used, Fourier analyzing the normal  $\xi^\psi = \vec{\xi} \cdot \vec{\nabla} \psi_T$ , binormal  $\eta = \vec{\xi} \cdot (\vec{\nabla} \theta - t \vec{\nabla} \phi)$  and parallel  $\mu = -\sqrt{g} \vec{\xi} \cdot \vec{\nabla} \phi$  components of the displacement away from the (up/down symmetric) equilibrium as:  $\xi^\psi = \sum_l \xi_l(\psi_T) \sin(m_l \theta - n \phi)$ ,  $\eta = \sum_l \eta_l(\psi_T) \cos(m_l \theta - n \phi)$ ,  $\mu = \sum_l \mu_l(\psi_T) \cos(m_l \theta - n \phi)$ , in terms of a toroidal mode number  $n$  and of a superposition of poloidal mode numbers  $m_l$ . Only the normal displacement  $\xi^\psi$  must be continuous [5] at the separatrix between the three tori and the surrounding spheromak, whereas the binormal  $\eta$  and the tangential  $\mu$  components can make jumps at  $\psi_T=\psi_T^x$ .

The poloidal angle  $\theta$  makes an excursion  $[0, \theta_X]$  on the lower outboard of the main spherical torus and then runs in the range  $[\theta_X, 2\pi+\theta_X]$  in the lower secondary torus. It continues to the angles  $[2\pi+\theta_X, 4\pi-\theta_X]$  on the inboard of the main spherical torus, runs in the range  $[4\pi-\theta_X, 6\pi-\theta_X]$  in the upper secondary torus and finishes its run in the range  $[6\pi-\theta_X, 6\pi]$  on the upper outboard of the main spherical torus. Inside the surrounding spheromak it simply covers the range  $[0, 2\pi]$ . So a correspondence is established between the poloidal angle  $\theta$  inside the tori and the poloidal angle  $3 \cdot \theta$  inside the surrounding spheromak. MHD "global" modes, which exist over the whole plasma, must have periodical perturbed displacements in terms of the Boozer poloidal angle  $\theta$ . For these global modes the allowed poloidal mode numbers can be all the integers  $m_l = \dots, -2, -1, 0, 1, 2, 3, \dots$  inside the main and the secondary tori.

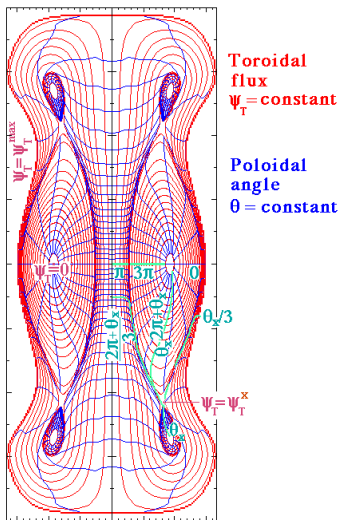


Fig. 3. Boozer coordinates of CKF force-free field with superposition constant  $\gamma=0.55$ .

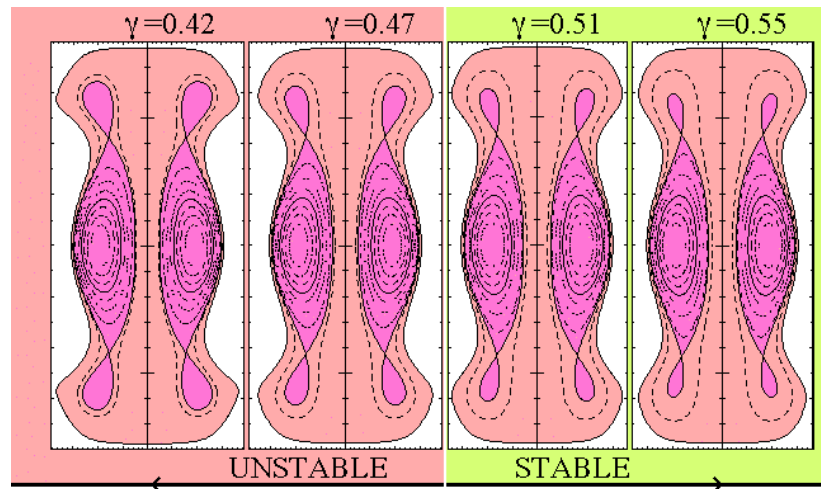


Fig. 4. Sequence of CKF force-free fields with ideal MHD stability boundary as a function of the superposition parameter  $\gamma$ .

Therefore the continuity of  $\xi^\Psi$  forces the allowed poloidal mode numbers of the global perturbations to be limited to multiples of 3:  $m_l = \dots, -6, -3, 0, 3, 6, 9, \dots$  inside the surrounding spheromak. However other MHD "internal" modes can still exist if their radial extent is limited to the surrounding spheromak,  $\psi_T^x \leq \psi_T \leq \psi_T^{\max}$ . These internal perturbations have as allowed poloidal mode numbers all the integers  $m_l = \dots, -2, -1, 0, 1, 2, 3, \dots$ , also inside the surrounding spheromak: their radial  $\xi^\Psi$  component will go to zero at the magnetic separatrix,  $\xi^\Psi(\psi_T^x) = 0$ , and will not have to match any requirement of periodicity on the three tori. The result of the ideal MHD stability calculations for low toroidal mode numbers ( $n=1, 2, 3$ ), assuming fixed boundary conditions at the edge of the plasma:  $\xi^\Psi(\psi_T^{\max}) = 0$ , is that the Chandrasekhar-Kendall-Furth force-free fields are stable when the value of the superposition parameter is greater than  $\gamma=0.5$  (see Fig. 4).

### 3. Unrelaxed Finite $\beta$ Configurations Similar to CKF Force-Free Fields

However force-free fields have  $\bar{\nabla}p=0$  and are so unable to confine plasmas of fusion interest. Nevertheless a variety of unrelaxed ( $\bar{\nabla}\mu \neq 0$ ,  $\bar{\nabla}p \neq 0$ ) MHD fixed boundary equilibria, similar in shape and topology to the CKF force-free fields, can be calculated (see Fig. 5).

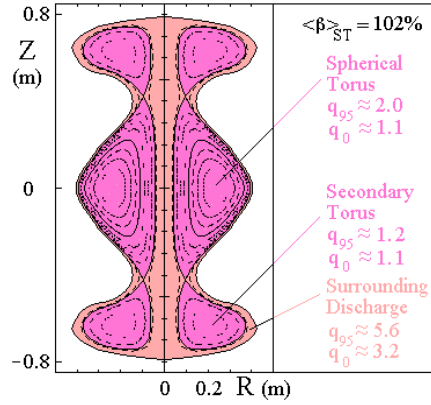


Fig. 5. Flux function contours of an unrelaxed CKF equilibrium, with  $\langle \beta \rangle_{ST} = 102\%$ .

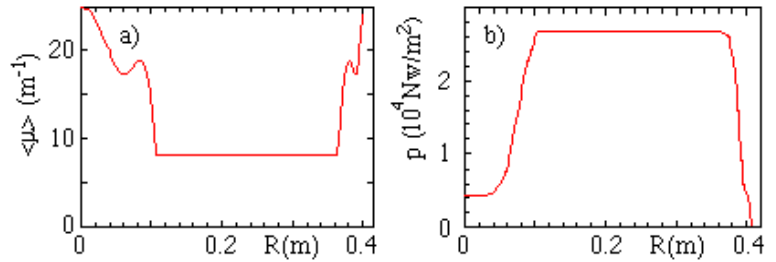


Fig. 6. Profiles of: a) surface averaged  $\langle \mu \rangle$ ; b) pressure, on the equator of the unrelaxed  $\langle \beta \rangle_{ST} = 102\%$  CKF of Fig. 5.

They have  $\mu = \mu_0 \bar{\mathbf{j}} \cdot \bar{\mathbf{B}} / B^2 = \text{constant}$  only at the edge of the plasma ( $\psi_T = \psi_T^{\max}$ ), as a boundary condition for the MHD equilibrium. In the example shown in Fig. 6 the surface averaged value  $\langle \mu \rangle = \mu_0 \langle \bar{\mathbf{j}} \cdot \bar{\mathbf{B}} / B^2 \rangle$  drops from  $\langle \mu \rangle = 25$ , at the edge of the surrounding spheromak, down to  $\langle \mu \rangle = 8$ , on the axis of the main spherical torus. Such  $\langle \mu \rangle$  profile corresponds to a sustainment obtained by driving current on the flux surfaces of the surrounding spheromak. Magnetic helicity, flowing down the  $\langle \mu \rangle$  gradient, will be injected into the main spherical torus, through magnetic reconnections at the X-points. The gradient of the pressure profile will presumably be concentrated in the same region where the gradient of  $\langle \mu \rangle$  has the largest variation (see Fig. 6).

The case shown in Figs. 5 and 6 corresponds to a very high beta in the spherical torus  $\beta_{ST} = 2\mu_0 \langle p \rangle_{ST} / \langle B^2 \rangle_{ST} = 102\%$ . It refers to an unrelaxed CKF in which the total poloidal spheromak current is  $I_e = 60$  kA, whereas the total toroidal current in the main spherical torus is  $I_p = 451$  kA. The safety factor of the main spherical torus has the values  $q_0^{ST} \sim 1.1$  on the magnetic axis and  $q_{95}^{ST} \sim 2.0$  at the edge (95% of the poloidal flux of the magnetic separatrix).

The two secondary torii, present on top and bottom of the main torus, have  $q_0^{SC} \sim 1.1$  on their magnetic axes and  $q_{95}^{SC} \sim 1.2$  at their edges. The spheromak discharge, which surrounds the three torii, has a larger safety factor: respectively  $q_0^P \sim 3.2$  on the symmetry axis and  $q_{95}^P \sim 5.6$  at the separatrix. An unrelaxed CKF equilibrium can be defined as a spherical torus ( $q_0^{ST} \sim 1, q_{95}^{ST} \sim 2$ ) enclosed within a spheromak endowed with high elongation and therefore with high winding number ( $q_0^P \sim 3, q_{95}^P \sim 5$ ) and intrinsically reversed shear.

The ideal MHD stability of the unrelaxed CKF configurations has been calculated with the same numerical code used for the CKF force-free fields. The ideal MHD stability of unrelaxed CKF equilibria, with profiles of  $\langle \mu \rangle(\psi_T)$  and  $p(\psi_T)$  similar to the ones shown in Fig. 6, has been calculated with fixed boundary conditions at the edge:  $\xi^\Psi(\psi_T^{\max})=0$ . Cases, like the one shown in Fig. 5, with a beta of the main spherical torus  $\beta_{ST}$  even exceeding unity, remain stable to all the ideal MHD perturbations with low toroidal mode numbers ( $n=1,2,3$ ), see Fig. 7. However with different choices of the  $\langle \mu \rangle$  and  $p(\psi)$  profiles (in particular with an excessive  $\Delta \langle \mu \rangle$  across the plasma), the beta limit with respect to low- $n$  ideal MHD modes is reduced. A case of an unstable CKF configuration with  $\beta_{ST}=67\%$  is shown in Fig. 8.

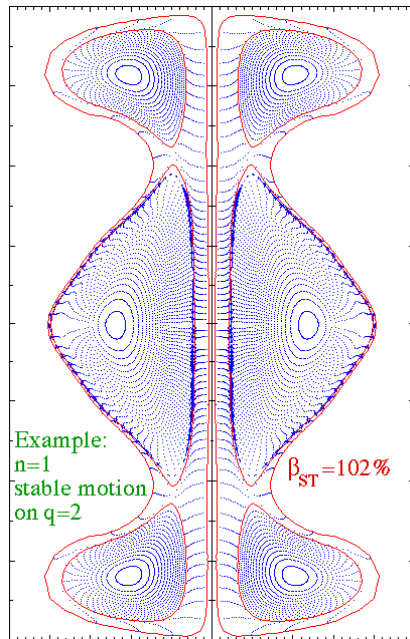


Fig. 7. Fluid displacement plot of a stable CKF.

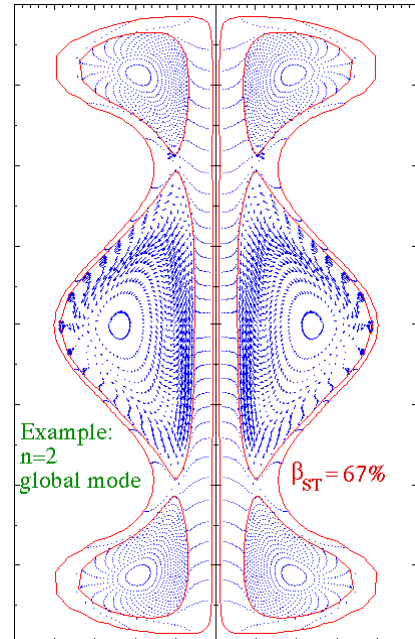


Fig. 8. Fluid displacement plot of an unstable CKF.

Reactor extrapolations of unrelaxed CKF magnetic configurations, endowed with the right helicity injection,  $\beta$  limit and energy confinement, will allow for an unimpeded outflow of the high energy charged fusion products. The charged fusion products will drift across the magnetic separatrix, to the degenerate X-points ( $B=0$ ) on top/bottom of the configuration easing direct energy conversion and the use of the burner as a space thruster.

## REFERENCES

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